

Introduction and Motivation

Classify accelerator waste for unrestricted release

- Many materials subject to potential activation
- Provide cost-efficient option for waste disposal
- Lower the overall dose within facility

Mission Relevance

Support accelerator operations in safeguards research

- Effective waste management within facilities
- Compare physics in radiation transport codes
- Knowledge of measurement environments

Technical Approach

- Find target thickness using ion range code SRIM
- 2. Simulate various source-target configurations to get neutron yields from PHITS, FLUKA, and MCNP
- 3. Obtain activation profiles with each code using beam current of 10 pnA and irradiation time of 1,000 hours and compare to ANSI N13.12 limits



Physics: JQMD + GEM **Yields Tally:** T-Cross, normalize by r² Activation Tally: T-DChain

<u>FLUKA:</u>

Physics: BME (PHYSICS 1.0 & 3.0) Yields Tally: USRYIELD Activation Tally: RADDECAY, IRRPROFI, DCYTIMES, RESNUCLEI, DCYSCORE

<u>MCNP:</u> Physics: LAQGSM + CEM (LCA 7j -211) **Yields Tally:** F2, normalize by r² Activation Tally: F4, execute CINDER90





Characterizing Activated Materials for Free Release at a Low-Energy Heavy-Ion Accelerator Facility Jordan D. Noey Ph.D. Candidate, University of Michigan Sunil Chitra¹, Kimberlee J. Kearfott² 1. Argonne National Lab, 2. University of Michigan

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